IN THE UNITED STATES PATENT AND TRADEMARK OFFICE

In re: application of:	)	Group Art Unit 3641
KIRKLAND D. BROACH et al.	)	Examiner: D.L. Greene Jr.
Serial No. 10/751,349	)	Entitled:
Filed: January 5, 2004	)	NUCLEAR FUEL ASSEMBLY DEBRIS FILTER BOTTOM
Attorney Docket No. ARF 2004-003	)	NOZZLE

August 30, 2005

Eckert Seamans Cherin & Mellott, LLC
600 Grant Street
Pittsburgh, PA 15219

MAIL STOP RCE
Commissioner for Patents
P.O. Box 1450
Alexandria, VA 22313

Declaration Under 37 CFR 1.132

Sir:

As a supplement to the Request for Continuing Examination and the Preliminary Amendment that are filed concurrently herewith, the following Declaration is offered:

I. Michael Y. Young declare and state:

1. That I received my Bachelor of Science degree in Mechanical Engineering from Rensselaer Polytechnic Institute at Troy, New York.

2. That I received my Master of Science degree in Mechanical Engineering from Rensselaer Polytechnic Institute .

3. That I worked for Westinghouse Electric Company LLC for 33 years.

4. That as an employee of Westinghouse Electric Company LLC, I currently hold the position of Chief Engineer.

5. That my curriculum vitae outlining the rest of my credentials is attached hereto.

6. That I have read the amended Claim 1 submitted with the amendment that is to accompany this Declaration, the Examiners rejection and the references the Examiner relied on.

7. That I am familiar with the Shallenberger design set forth in U.S. Patent No. 4,900,507, which is assigned to his employer, Westinghouse Electric Company LLC.

8. That the Shallenberger design and the teaching of the Shallenberger patent do not contemplate having a flared outlet at the fuel assembly bottom nozzle coolant flow holes nor does the Shallenberger patent describe or contemplate having a double chamfered inlet, both of which are specified in the amended Claim 1 to be filed concurrently herewith.

9. That the Shallenberger patent was published in 1990 and to the best of my knowledge, information and belief, except for the inventors of the subject application, no one has employed or suggested the use of a double chamfered inlet and flared outlet for the coolant holes in a fuel assembly bottom nozzle, since the publication of the Shallenberger patent.

10. That the double chamfered inlet and flared outlet is a novel way to achieve pressure drop reduction near that of a rounded inlet and venturi outlet while meeting numerous constraints imposed by the performance requirements of the nuclear fuel assembly, such as maintenance of uniform flow, and by the manufacturing process, such as closely spaced flow holes and accommodation of tolerances.

11. That none of the references relied upon by the Examiner suggest that a venturi profile for the coolant holes in the bottom nozzle of a fuel assembly will not adversely impact the other coolant criteria that need to be fulfilled for satisfactory coolant transport through the nuclear core.

12. That the double chamfered inlet has been found not to adversely impact the benefit of the venturi profile in the bottom nozzle coolant flow holes, but provides a significant manufacturing savings over the normal venturi gradient profile between the inlet and outlet of the venturi flow holes.

I further declare and state that all statements made herein of my own knowledge are true and that all statements made on information and belief are believed to be true; and further, that these statements were made with the knowledge that willful false statements and the like so made are punishable by fine or imprisonment, or both, under Section 1001 of Title 18 of the United States Code, and that such willful false statements may jeopardize the validity of the application or any patent issuing thereon.

Respectfully submitted,

Michael A. Young, Declarant **Date** August 30, 2005

RESUME

Michael Y. Young
January 1, 2002

ADDRESS:

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EDUCATION:

BS - Acronautical Engineering, Rensselaer Polytechnic Institute, Troy, New York. 1966 - 1970

MS - Mechanical Engineering, Rensselaer Polytechnic Institute, Troy, New York. 1970 - 1972

EXPERIENCE:

2001- 2005: Chief Engineer, Nuclear Fuels

Currently responsible for:

1. Identification of Technological Opportunities for Nuclear Fuels: Lead the process to identify areas where technological opportunities exist, which could produce significant breakthroughs. Utilize this knowledge to focus the organization's technology, research, and development activities.
2. Technical Knowledge Management: As appropriate, ensure that there is continuous organizational leveraging and integration of historical technological solutions and new research & development activities, including:
    a. Coordination of Development Programs
    b. Industry interactions to assure consistent positions and approach
    c. Foster key University relationships
    d. Review and solicit key technical papers and establish strategy for attending conferences
3. Ensure an organizational understanding of the unmet needs of the nuclear fuel customer (operational, licensing, margins, etc.. Develop actions/plans to address these needs.
4. Facilitate the allocation of funding for development projects, oversee completion of those activities, and monitor progress.
5. Technological Problem Resolution – Where appropriate, lead and/or facilitate technological review teams to review critical technical issues, new designs and services, etc. Serve as the technical final solution in areas of technical disputes and coordinate resolution of these issues. Lead department task teams addressing critical high impact

issues involving multi-discipline functions.

6. Key Strategic Initiatives – Where appropriate & assigned, lead the implementation of key strategic initiatives which develop high value for customers and NFBU. Interface with the Strategy Department to provide key technical strategy input.

1993-2001: Consulting Engineer, Nuclear Services and Nuclear Fuels.

Led team developing Next Generation Fuel grid.

Led team to resolve the V5H grid to rod fretting issue. Outcome was development of the RFA-2 grid design and a more complete understanding of fretting phenomena that should prevent problems in future.

Led multi-division team to resolve the Axial Offset Anomaly issue. Role included defining test and development programs, interactions with customers and EPRI, and development of predictive tool to determine AOA risk of core designs. Outcome was the BOB code, increased understanding of crud deposition and AOA process, and an influential position in the EPRI/Utility AOA working group.

Led team developing the safety case for the AP600 advanced reactor design. Prepared strategy for making presentations to the Advisory Committee on Reactor Safeguards, a key obstacle to approval of the AP600 design.

Led team of experts in 4 year project to develop new computer technology for nuclear safety analysis ("Best Estimate" Loss of Coolant Accident Evaluation Model). Project involved solution of numerous problems in thermal-hydraulics, numerical methods, and probability, and continual interaction and negotiation with Nuclear Regulatory Commission (NRC) and Advisory Committee on Reactor Safeguards (ACRS). Successful licensing of methodology (the first of its kind in the industry) required high degree of credibility in discussions with NRC and ACRS.

1991-1993: Manager, Nuclear Safety Analysis, Westinghouse Energy Systems Business Unit.

Managed technical staff of 6 managers and approximately 100 engineers engaged in safety and engineering analysis. Group was responsible for sales of approximately ten to fifteen million dollars per year. Responsibilities included interactions with customers and NRC to resolve licensing and technical issues, including application of new reporting regulations for computer code changes, and application of 10CFR50.59 safety evaluation process.

1981-1991: Manager, Thermal Hydraulic Testing and Analysis, Westinghouse Energy Systems Business Unit.

Initiated program to develop an advanced Loss of Coolant Accident (LOCA) analysis technology, using statistical methodology.

Managed group of 15 engineers responsible for designing and overseeing the performance of several test and analysis programs, including the FLECHT reflood heat transfer and MB-2 steam generator programs.

Worked with Westinghouse Owner's Group (WOG) to resolve several key issues such as quantifying the extent of radiation release in steam generator tube rupture.

Helped resolve the clad ballooning issue that was a key obstacle in licensing PWRs in the UK.

Successfully obtained approval by NRC for application of improved safety analysis methods (BASH code). These methods allowed Westinghouse to supply new nuclear fuel designs with higher peaking factors than the competition.

1972-1981: Engineer, Westinghouse Nuclear Energy Systems.

Began Westinghouse career as associate engineer in 1972. Helped develop Westinghouse Appendix K LOCA model. Was lead engineer in effort to license Upper Head Injection system for Sequoyah Nuclear Plant. Developed first significant improvement to the Appendix K model for increased LOCA margin (the BART code). Work required frequent presentations and negotiations with NRC staff and other oversight committees.

OUTSIDE ACTIVITIES:

1987, 1988 Commissioner, Little League Baseball
1986, 1987, 1988 Coach, Little League Baseball
1985-1989, Member and president of neighborhood pool committee

AWARDS:

Division Engineering awards in 1980, 1984, 1987, 1989, 1995, 2000
George Westinghouse company award, 2001.

PAPERS:

1. "A Mechanistic Model for the Best Estimate Analysis of Reflood Transients (The BART Code)", 19th National Heat transfer Conference, Orlando, Florida, 1980.
2. "Advances in PWR LOCA Analysis", Technical Workshop on Nuclear Reactor Safety and Thermo-hydraulics, Seoul, Korea, 1985.
3. "An Implicit Method to Speed up WCOBRA/TRAC", Nuclear Science and Engineering, 1988.
4. "Application of Realistic Thermal-Hydraulic Methods for Pressurized Water Reactors with Upper Plenum Injection", Nuclear Technology, 1989.
5. "Development of LOCA Margin for Two-Loop PWR", Nuclear Plant Journal, January, 1989.

6. "Downflow Two-Phase Pressure Drop in Irregular Channels with Plates", Int. Symposium on Gas-liquid Two Phase Flows, ASME Winter Annual Meeting, 1990.
7. "Application of PWR LOCA Margin with the Revised Appendix K Rule", Nuclear Engineering and Design, 1992.
8. "Application of code scaling and uncertainty methodology to the large break loss of coolant", Nuclear Engineering and Design, 1998
9. "Assessment of flooding in a best estimate thermal hydraulic code (WCOBRA/TRAC)", Nuclear Engineering and Design, 1998.
10. "Best Estimate Analysis of the large break loss of coolant accident", ICONE-6, 1998
11. "Flooding in the pressurizer surge line of AP600 plant and analyses of APEX data", Nuclear Engineering and Design, 1999.
12. "Direct contact heat transfer model for dispersed flow film boiling", Nuclear Technology, December 2000.
13. "The flow field in a reactor core and its effect on rod vibration and wear", ASME Pressure Vessels and Piping Conference, July 2001.
14. "Flow Induced Vibration and Fretting Wear in PWR Fuel", ICONE 10, April 2002.
15. "The Effect Of Corrosion Product Deposition On Fuel Management", Advances in Nuclear Fuel Management III (ANFM 2003), October 5-8, 2003.
16. "A Comprehensive Method for Assessing Fuel Performance Risks Due to Crud Deposition", Proceedings of the 2004 International Meeting on LWR Fuel Performance, Orlando, Florida, September 19-22, 2004.